



U.S. Department  
of Transportation

Pipeline and  
Hazardous Materials  
Safety Administration

East Building, PHH-23  
1200 New Jersey Ave, SE  
Washington, D.C. 20590

**COMPETENT AUTHORITY CERTIFICATION FOR A  
TYPE B(U)  
RADIOACTIVE MATERIALS PACKAGE DESIGN  
CERTIFICATE USA/6613/B(U)-96, REVISION 23**

The Competent Authority of the United States certifies that the radioactive material package design described in this certificate satisfies the regulatory requirements for a Type B(U) package as prescribed in the regulations of the International Atomic Energy Agency<sup>1</sup> and the United States of America<sup>2</sup> The package design is approved for use within the United States for import and export shipments made in accordance with applicable international and domestic transport regulations.

1. Package Identification - Model 702.
2. Package Description and Authorized Radioactive Contents - as described in U.S. Nuclear Regulatory Commission Certificate of Compliance No. 6613, Revision 22 (attached).
3. General Conditions -
  - a. Each user of this certificate must have in his possession a copy of this certificate and all documents necessary to properly prepare the package for transportation. The user shall prepare the package for shipment in accordance with the documentation and applicable regulations.
  - b. Each user of this certificate, other than the original petitioner, shall register his identity in writing to the Office of Engineering and Research, (PHH-23), Pipeline and Hazardous Materials Safety Administration, U.S. Department of Transportation, Washington D.C. 20590-0001.

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<sup>1</sup> "Regulations for the Safe Transport of Radioactive Material, 2012 Edition, No. SSR-6" published by the International Atomic Energy Agency (IAEA), Vienna, Austria.

<sup>2</sup> Title 49, Code of Federal Regulations, Parts 100-199, United States of America.

**CERTIFICATE USA/6613/B(U)-96, REVISION 23**

- c. This certificate does not relieve any consignor or carrier from compliance with any requirement of the Government of any country through or into which the package is to be transported.
  - d. Records of Management System activities required by Paragraph 306 of the IAEA regulations<sup>1</sup> shall be maintained and made available to the authorized officials for at least three years after the last shipment authorized by this certificate. Consignors in the United States exporting shipments under this certificate shall satisfy the applicable requirements of Subpart H of 10 CFR 71.
4. Marking and Labeling - The package shall bear the marking USA/6613/B(U)-96 in addition to other required markings and labeling.
5. Expiration Date - This certificate expires on February 29, 2028. Previous editions which have not reached their expiration date may continue to be used.

This certificate is issued in accordance with paragraph(s) 810 of the IAEA Regulations and Section 173.471 of Title 49 of the Code of Federal Regulations, in response to the November 9, 2022 petition by QSA Global, Inc., Burlington, MA, and in consideration of other information on file in this Office.

Certified By:



William Schoonover  
Associate Administrator for Hazardous  
Materials Safety

December 08, 2022  
(DATE)

Revision 23 - Issued to endorse U.S. Nuclear Regulatory Commission Certificate of Compliance No. 6613, Revision 22.

**CERTIFICATE OF COMPLIANCE  
FOR RADIOACTIVE MATERIAL PACKAGES**

1.	a. CERTIFICATE NUMBER 6613	b. REVISION NUMBER 22	c. DOCKET NUMBER 71-6613	d. PACKAGE IDENTIFICATION NUMBER USA/6613/B(U)-96	PAGE 1	PAGES OF 3
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2. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10, *Code of Federal Regulations*, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION

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| a. ISSUED TO ( <i>Name and Address</i> )<br>QSA Global Inc.<br>40 North Avenue<br>Burlington, MA 01803 | b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION<br>QSA Global Inc., application dated<br>December 4, 2017. |
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4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

5.

(a) Packaging

- (1) Model No.: 702
- (2) Description

The Model No. 702 is composed of a stainless steel cylinder shield cask assembly secured to a rectangular carbon steel skid and further covered by a carbon steel cage assembly that is bolted to the skid at each corner. The overall dimensions of the Model No. 702 are 19 ¾" x 21" x 19" (502 mm x 533 x 483 mm) and the maximum weight is 410 pounds (186 kg) including contents. The shield cask assembly contains a depleted uranium shield and a cover assembly sealed by a neoprene gasket. The cover assembly flange is anchored to the cask with six bolts.

There is no locking assembly on the Model No. 702. Sources are secured in the shielded position by the cover assembly and two of the six securing bolts of the cover assembly are seal-wired with a tamper indicator seal. Metallic canisters and inserts used for holding special form sources are limited to non-pyrophoric metals with a melting temperature at or above 800°C.

- (3) Drawings

The Model No. 702 and other system components are constructed in accordance with QSA Global Drawing No. R70290, sheets 1 to 9, Revision AA.

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5.(b) Contents

(1) Type and form of material

Iridium-192, Selenium-75, Cesium-137, and Ytterbium-169 as special form sealed sources.

(2) Maximum quantity of material per package:

<u>Isotopes</u>	<u>Content Activity</u>
Cs-137	500 Ci (18.5 TBq)
Se-75	10,000 Ci (370 TBq)
Yb-169	10,000 Ci (370 TBq)
Ir-192	15,000 Ci (555 TBq)

<u>Isotope</u>	<u>Output Activity</u>
Ir-192	6,500 Ci (240.5 TBq)**

\*\*Ir-192 sources measured in Output Activity are cylindrical with steel encapsulations. Source configuration dimensions at the time of output activity determination are not to exceed:

3mm diameter (Ir-192)  
4mm height (Ir-192)  
1.675mm encapsulation wall thickness

OR

2.7mm diameter (Ir-192)  
5.25mm height (Ir-192)  
1.825mm encapsulation wall thickness

Additional encapsulations may be added so long as the output activity determination was made on an inner source configuration meeting the dimensions above. Additional encapsulation metallic inserts/spacers may be added that exceed the encapsulation thicknesses above as long as the total dimensions (Ir-192 material in source + encapsulation) do not exceed that of the above specified sources (i.e., the Ir-192 source dimensions are decreased by the amount the encapsulation is increased). Additional metallic encapsulation in excess of the maximum dimensions stated above need not be steel as long as density is equal to or less than that of Ir-192.

Output curies are determined by measuring the source output at 1 meter from the device and expressing its activity in curies. (Procedures reference: American National Standards Institute N432-1980, "Radiological Safety for the Design and Construction of Apparatus for Gamma Radiography.")

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5.(b) Contents (continued)

(3) Maximum decay heat per package: 92 watts

(4) Maximum weight of contents: 0.88 pounds (400 grams)

6. The name plate must be fabricated of material capable of resisting the fire test of (10 CFR) Part 71 and maintaining their legibility.

7. In addition to the requirements of Subpart G of Title 10 of the *Code of Federal Regulations*, 10 CFR Part 71:

(a) Each package shall be operated and prepared for shipment in accordance with Chapter 7 of the application, as supplemented.

(b) The package must meet the Acceptance Tests and Maintenance Program of Chapter 8 of the application, as supplemented.

8. Revision No. 20 and Revision No. 21 of this certificate may be used until February 28, 2023.

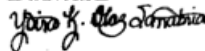
9. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR 71.17.

10. Expiration Date: February 29, 2028.

REFERENCES

QSA Global Inc., application dated December 4, 2017, as supplemented on August 8, 2018, May 23, 2022, and June 22, 2022

FOR THE U.S. NUCLEAR REGULATORY COMMISSION



Signed by Diaz-Sanabria, Yoira  
on 10/31/22

Yoira Diaz Sanabria, Chief  
Storage and Transportation Licensing Branch  
Division of Fuel Management  
Office of Nuclear Material Safety  
and Safeguards

Date: See digital signature



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Transportation

**Pipeline and  
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East Building, PHH-23  
1200 New Jersey Ave, SE  
Washington, D.C. 20590

**CERTIFICATE NUMBER:** USA/6613/B(U)-96

**ORIGINAL REGISTRANT(S) :**

QSA Global, Inc.  
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Burlington, MA, 01803  
USA

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U.S. Department of Energy  
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USA

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